Indexes to *Nuclear Safety*, Volume 37

Cumulative Keyword in Context (KWIC) indexes of issues of *Nuclear Safety* through Volume 22 were published as separate documents and are available from the National Technical Information Service. Starting with Volume 23 (1982), author and KWIC indexes are published in the first issue of the following volume. This issue contains these indexes for *Nuclear Safety*, Volume 37. Both indexes use a six-digit number to indicate the location of the indexed material. The six-digit number is divided into four parts (00-0-000), which stand, respectively, for volume-number-page. The authors are indexed alphabetically. In the KWIC index, the article titles are permuted around the various significant words contained therein. For example, the title "A Scheme for Passive Isolation of the Containment of a Reactor," is indexed under the words *Passive*, *Isolation*, *Containment,* and *Reactor*. The index words are arranged alphabetically in a column in the center of the page with the titles permuted around them. A slash (/) indicates the end of a title. The two indexes follow.

KWIC Index

1994 ACCIDENT SEQUENCE PRECURSOR PROGRAM RESULTS/ 37-1-73
A REGULATORY ASSESSMENT OF TEST DATA FOR REACTIVITY-INITIATED ACCIDENTS/ 37-4-271
TWENTY-FOURTH DOE/NRC AIR-CLEANING AND TREATMENT CONFERENCE/ 37-3-211
VULNERABILITY OF MULTIPLE- BARRIER SYSTEMS/ 37-2-139
SYSTEM FOR MAINTENANCE MANAGEMENT OF A BOILING-WATER REACTOR POWER PLANT/ 37-1-1
ONS/ FRENCH STUDIES ON HIGH- BURNUP FUEL TRANSIENT BEHAVIOR UNDER RIA CONDITI 37-4-289
NSRR/RIA EXPERIMENTS WITH HIGH- BURNUP PWR FUELS/ 37-4-328
TION EXPERIMENTS ON INTERMEDIATE- AND HIGH- BURNUP TEST RODS/ 37-4-372
RESIN BY HYDROGEN PEROXIDE/ A STUDY OF WET CATALYTIC OXIDATION OF RADIOACTIVE SPENT ION EXCHANGE 37-2-149
EEN EMERGENCY RESPONSE AND SAFETY ANALYSIS CODES USED AT THE SAVANNAH RIVER SITE/ 37-2-157
THE NUCLEAR COMMUNITY AND THE PUBLIC COGNITIVE AND CULTURAL INFLUENCES ON THINKING AB 37-2-97
RY PARAMETERS AND METHODS FOR DETERMINING CONDENSATION HEAT TRANSFER TO CONTAINMENT 37-1-26
TWENTY -FOURTH DOE/NRC AIR-CLEANING AND TREATMENT CONFERENCE/ 37-3-211
DETERMINING CONDENSATION HEAT TRANSFER TO CONTAINMENT STRUCTURES/ AN OVERVIEW OF THE 37-1-26
ASSESSMENT OF SPENT FUEL COOLING/ 37-3-237
COMMUNITY AND THE PUBLIC COGNITIVE AND CULTURAL INFLUENCES ON THINKING ABOUT NUCLEAR RI 37-2-97
OUS FACILITIES/ MODERN TORNADO DESIGN OF NUCLEAR AND OTHER POTENTIALLY HAZARD 37-1-49
ATTITUDES AND PRACTICES REGARDING DISPOSAL OF LIQUID NUCLEAR WASTE AT CLINTON LABOR 37-3-181
CON FERENCE/ TWENTY-FOURTH DOE/NRC NUCLEAR AIR-CLEANING AND TREATMENT 37-3-202
STRUCTURE/ OFF-SITE NUCLEAR EMERGENCY MANAGEMENT IN GERMANY UNDER THE AU 37-3-202
TUDY AND RESOLUTION OF DIFFERENCES BETWEEN EMERGENCY RESPONSE AND SAFETY ANALYSIS CODES US 37-2-157
TEST RODS/ ASSESSMENT OF RIA-SIMULATION EXPERIMENTS ON INTERMEDIATE- AND HIGH-BURNUP 37-4-372
NSRR/RIA EXPERIMENTS WITH HIGH-BURNUP PWR FUELS/ 37-4-328
APRIL 6, 1993/ EXPLOSION IN THE TOMSK-7 REPROCESSING PLANT ON 37-3-222
OF NUCLEAR AND OTHER POTENTIALLY HAZARDOUS FACILITIES/ MODERN TORNADO DESIGN 37-1-49
AVIOR UNDER RIA CONDITIONS/ FRENCH STUDIES ON HIGH-BURNUP FUEL TRANSIENT BEH 37-4-289
FRENCH STUDIES ON HIGH-BURNUP FUEL TRANSIENT BEHAVIOR UNDER RIA CONDITIONS/ 37-4-289
ASSESSMENT OF SPENT FUEL COOLING/ 37-3-237
NSRR/RIA EXPERIMENTS WITH HIGH-BURNUP PWR FUELS/ 37-4-328
OFF-SITE NUCLEAR EMERGENCY MANAGEMENT IN GERMANY UNDER THE AUSPICES OF THE FEDERAL STRUC 37-3-202
MODERN TORNADO DESIGN OF NUCLEAR AND OTHER POTENTIALLY HAZARDOUS FACILITIES/ 37-1-49
AND METHODS FOR DETERMINING CONDENSATION HEAT TRANSFER TO CONTAINMENT STRUCTURES/ 37-1-26
NDITIONS/ FRENCH STUDIES ON HIGH-BURNUP FUEL TRANSIENT BEHAVIOR UNDER RIA CO 37-4-289
NSRR/RIA EXPERIMENTS WITH HIGH-BURNUP PWR FUELS/ 37-4-328
IMULATION EXPERIMENTS ON INTERMEDIATE- AND HIGH-BURNUP TEST RODS/ 37-4-372
TON LABORATORIES IN THE VERY EARLY YEARS: A HISTORICAL ANALYSIS/ ATTITUDES AND PRACTICES 37-3-181
THE IGNITION POTENTIAL OF URANIUM METAL AND HYDRIDE/ ON PREDICTION OF 37-1-12
ON PREDICTION OF THE IGNITION POTENTIAL OF URANIUM METAL AND HYDRIDE/ 37-1-12

UNITY AND THE PUBLIC COGNITIVE AND CULTURAL
ASSESSMENT OF RIA-SIMULATION EXPERIMENTS ON
ET CATALYTIC OXIDATION OF RADIOACTIVE SPENT
DISPOSAL OF LIQUID NUCLEAR WASTE AT CLINTON
ANALYSIS OF A PWR
TITUDES AND PRACTICES REGARDING DISPOSAL OF
POWER PLANT/ A DECISION SUPPORT SYSTEM FOR
A DECISION SUPPORT SYSTEM FOR MAINTENANCE
TWENTY-THIRD WATER REACTOR SAFETY INFORMATION
ON PREDICTION OF THE IGNITION POTENTIAL OF URANIUM
TWENTY-FOURTH DOE/
TWENTY-FOURTH DOE/NRC
IES/ MODERN TORNADO DESIGN OF
THINKING ABOUT NUCLEAR RISK/ THE
OF THE FEDERAL STRUCTURE/ OFF-SITE
AND CULTURAL INFLUENCES ON THINKING ABOUT
S AND PRACTICES REGARDING DISPOSAL OF LIQUID
AUSPICES OF THE FEDERAL STRUCTURE/
PEROXIDE/ A STUDY OF WET CATALYTIC
CTIVE SPENT ION EXCHANGE RESIN BY HYDROGEN
NAGEMENT OF A BOILING-WATER REACTOR POWER
MANAGEMENT OF A BOILING-WATER REACTOR
THE NUCLEAR COMMUNITY AND THE
1994 ACCIDENT SEQUENCE
NSRR/RIA EXPERIMENTS WITH HIGH-BURNUP
ANALYSIS OF A
A STUDY OF WET CATALYTIC OXIDATION OF
A REGULATORY ASSESSMENT OF TEST DATA FOR
MAINTENANCE MANAGEMENT OF A BOILING-WATER
TWENTY-THIRD WATER
REACTIVITY-INITIATED ACCIDENTS/ A
EXPLOSION IN THE TOMSK-7
THE RUSSIAN RIA
OXIDATION OF RADIOACTIVE SPENT ION EXCHANGE
HIGH-BURNUP FUEL TRANSIENT BEHAVIOR UNDER
NSRR/
RESULTS/ THE RUSSIAN
HIGH-BURNUP TEST RODS/ ASSESSMENT OF
TURAL INFLUENCES ON THINKING ABOUT NUCLEAR
TION, EXECUTION, AND RESULTS/ THE
IFFERENCES BETWEEN EMERGENCY RESPONSE AND
TWENTY-THIRD WATER REACTOR
SPONSE AND SAFETY ANALYSIS CODES USED AT THE
ANALYSIS OF A PWR LBLOCA WITHOUT
ANALYSIS CODES USED AT THE SAVANNAH RIVER
CONDENSATION HEAT TRANSFER TO CONTAINMENT
VULNERABILITY OF MULTIPLE-BARRIER
EXPLOSION IN THE
Y HAZARDOUS FACILITIES/ MODERN
FRENCH STUDIES ON HIGH-BURNUP FUEL
TREATMENT CONFERENCE/
MEETING/
ON PREDICTION OF THE IGNITION POTENTIAL OF
CTICES REGARDING DISPOSAL OF LIQUID NUCLEAR
INFLUENCES ON THINKING ABOUT NUCLEAR RISK/ 37-2-97
INTERMEDIATE- AND HIGH-BURNUP TEST RODS/ 37-4-372
ION EXCHANGE RESIN BY HYDROGEN PEROXIDE/ 37-2-139
LABORATORIES IN THE VERY EARLY YEARS: A HISTORICA 37-3-181
LBLOCA WITHOUT SCRAM/ 37-2-126
LIQUID NUCLEAR WASTE AT CLINTON LABORATORIES IN T 37-3-181
MAINTENANCE MANAGEMENT OF A BOILING-WATER REAC 37-1-1
MANAGEMENT OF A BOILING-WATER REACTOR POWER 37-1-1
MEETING/ 37-2-109
METAL AND HYDRIDE/ 37-1-12
NRC NUCLEAR AIR-CLEANING AND TREATMENT CONFERE 37-3-202
NSRR/RIA EXPERIMENTS WITH HIGH-BURNUP PWR FUELS/ 37-4-328
NUCLEAR AIR-CLEANING AND TREATMENT CONFERENCE/ 37-3-211
NUCLEAR AND OTHER POTENTIALLY HAZARDOUS FACILIT 37-1-49
NUCLEAR COMMUNITY AND THE PUBLIC COGNITIVE AND 37-2-97
NUCLEAR EMERGENCY MANAGEMENT IN GERMANY UNDE 37-3-202
NUCLEAR RISK/ THE NUCLEAR COMMUNITY AND THE 37-2-97
NUCLEAR WASTE AT CLINTON LABORATORIES IN THE EAR 37-3-181
OFF-SITE NUCLEAR EMERGENCY MANAGEMENT IN GERMA 37-3-202
OXIDATION OF RADIOACTIVE SPENT ION EXCHANGE RESI 37-2-149
PEROXIDE/ A STUDY OF WET CATALYTIC OXIDATION 37-2-149
PLANT/ A DECISION SUPPORT SYSTEM FOR MAINTENANCE 37-1-1
POWER PLANT/ A DECISION SUPPORT SYSTEM FOR 37-1-1
PUBLIC COGNITIVE AND CULTURAL INFLUENCES ON THIN 37-2-97
PRECURSOR PROGRAM RESULTS/ 37-1-73
PWR FUELS/ 37-4-328
PWR LBLOCA WITHOUT SCRAM/ 37-2-126
RADIOACTIVE SPENT ION EXCHANGE RESIN BY HYDROGE 37-2-139
REACTIVITY-INITIATED ACCIDENTS/ 37-4-271
REACTOR POWER PLANT/ A DECISION SUPPORT SYSTEM 37-1-1
REACTOR SAFETY INFORMATION MEETING/ 37-2-109
REGULATORY ASSESSMENT OF TEST DATA FOR 37-4-271
REPROCESSING PLANT ON APRIL 6, 1993/ 37-3-222
RESEARCH PROGRAM: MOTIVATION, DEFINITION, AND RE 37-4-343
RESIN BY HYDROGEN PEROXIDE/ A STUDY OF 37-2-149
RIA CONDITIONS/ FRENCH STUDIES ON 37-4-289
RIA EXPERIMENTS WITH HIGH-BURNUP PWR FUELS/ 37-4-328
RIA RESEARCH PROGRAM: MOTIVATION, DEFINITION, AND 37-4-343
RIA-SIMULATION EXPERIMENTS ON INTERMEDIATE- AND 37-4-372
RISK/ THE NUCLEAR COMMUNITY AND THE PUBLIC 37-2-97
RUSSIAN RIA RESEARCH PROGRAM: MOTIVATION, DEFINI 37-4-343
SAFETY ANALYSIS CODES USED AT THE SAVANNAH RIVER 37-2-157
SAFETY INFORMATION MEETING/ 37-2-109
SAVANNAH RIVER SITE/ A COMPARISON STUDY AND 37-2-157
SCRAM/ 37-2-126
SITE/ A COMPARISON STUDY AND RESOLUTION OF 37-2-157
STRUCTURES/ AN OVERVIEW OF THE PRIMARY PAR 37-1-26
SYSTEMS/ 37-2-139
TOMSK-7 REPROCESSING PLANT ON APRIL 6, 1993/ 37-3-222
TORNADO DESIGN OF NUCLEAR AND OTHER POTENTIALL 37-1-49
TRANSIENT BEHAVIOR UNDER RIA CONDITIONS/ 37-4-289
TWENTY-FOURTH DOE/NRC NUCLEAR AIR-CLEANING AND 37-3-211
TWENTY-THIRD WATER REACTOR SAFETY INFORMATION 37-2-109
URANIUM METAL AND HYDRIDE/ 37-1-12
VULNERABILITY OF MULTIPLE-BARRIER SYSTEMS/ 37-2-139
WASTE IN THE VERY EARLY YEARS: A HISTORICAL ANALY 37-3-181

Author List

Almenas, K.	37-1-26	Luangdilok, W.	37-1-12
Asmolov, V.	37-4-343	Lussie, W. G.	37-3-222
Balourdet, M.	37-4-289	Macian-Juan, R.	37-2-126
Bayer, A.	37-3-202	Mahaffy, J. H.	37-2-126
Bellamy, R. R.	37-3-211	McCardell, R. K.	37-4-271
Belles, R. J.	37-1-73	Meyer, M. A.	37-2-97
Bittner, S.	37-3-202	Meyer, R. O.	37-4-271
Chung, H. M.	37-4-271	Minarick, J. W.	37-1-73
Cletcher, J. W.	37-1-73	Montgomery, R. O.	37-4-372
Copinger, D. A.	37-1-73	Nagase, F.	37-4-328
Copinger, D. A.	37-2-109	O'Reilly, P. D.	37-1-73
Diamond, D. J.	37-4-271	Ornstein, H. L.	37-3-237
Dolan, B. W.	37-1-73	Ozer, O.	37-4-372
Epstein, M.	37-1-12	Papin, J.	37-4-289
Fauske, H. K.	37-1-12	Plys, M. G.	37-1-12
First, M. W.	37-3-211	Porco, R. D.	37-3-211
Frizonnet, J. M.	37-4-289	Pullani, S. V.	37-3-237
Fujishiro, T.	37-4-328	Rashid, Y. R.	37-4-372
Fuketa, T.	37-4-328	Ray, A.	37-1-1
Green, J.	37-1-26	Schmitz, F.	37-4-289
Hayes, J. J.	37-3-211	Scott, H. H.	37-4-271
Hyder, M. L.	37-3-222	Shen, J. H.	37-1-1
Ibarra, J. G.	37-3-237	Simpkins, A. A.	37-2-157
Ishijima, K.	37-4-328	Stevenson, J. D.	37-1-49
Jian, X.	37-2-149	Stow, S. H.	37-3-181
Jones, W. R.	37-3-237	Tyler, T. N.	37-2-126
Korn, H.	37-3-202	Witmer, F. E.	37-3-222
Lamare, F.	37-4-289	Wu, T.	37-2-149
Lanik, G. F.	37-3-237	Yang, R. L.	37-4-372
Lemoine, F.	37-4-289	Yegorova, L.	37-4-343
Levine, S.	37-1-1	Yun, G.	37-2-149
Lind, N. C.	37-2-139	Zaho, Y.	37-1-49